Reactor Fuels

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Short Course: Introduction to Nuclear Fuel Cycle Chemistry

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Focus On:

- Characteristics relevant to separation of used nuclear reactor fuel into its constituent parts: nuclear fuel reprocessing
- Fuels most relevant to the U.S. and separations

Civilian nuclear power

Outline

- Metal clad fuels: oxide and variants
 - Description and fabrication
 - Oxide fuels are used in many reactors
 - Light-water reactors (LWRs)
 - Boiling Water Reactors (BWRs)
 - Pressurized Water Reactors (PWRs)
 - Fast Reactors cooled with Na, K, Bi, Pb, He
- Graphite-based fuels

- Description of fuels and how they are fabricated

- High-Temperature Gas-Cooled Reactors (HTGRs)
 - Also called Very-High-Temperature Reactors (VHTRs)
- Pebble-Bed Modular (Gas Cooled) Reactor (PBMR)
- Spent fuel characteristics

Metal-Clad Fuels

Pressurized Water Reactor (PWR)

- Dimensions: square, H=4.1m, 21cm x 21 cm
- Weight: 460 kgU, 520 kg UO₂, 135 kg hardware
 - Hardware mostly Zircaloy (Zr with Sn, Fe, Cr)
 - Grid spacers: Zircaloy, Inconel, stainless steel
 End pieces: Stainless steel, Inconel
- Fuel element array: 14 x 14 to 17 x 17
- Fuel element size: 1 cm OD, H=3.9m
- Enrichment: 3-5%
- May have separate burnable poison rods

Pressurized Water Reactor



Pressurized Water Reactor

Pressurized-Water Fuel Assembly



Boiling Water Reactor (BWR)

- Dimensions: square, H=4.5m, 14 cm x 14 cm
- Weight: 180 kgU, 210 kg UO₂, 110 kg hardware
 - Hardware mostly Zircaloy (Zr with Sn, Fe, Cr)
 - Grid spacers: Zircaloy
 - Channel (aka shroud): Zircaloy
 - End pieces: Stainless steel
- Fuel element array: 8 x 8
- Fuel element size: 1.25 cm OD, H=4.1m
- Enrichment: 2.5-4.5%
- May have Gd in some rods and variable enrichment in 3-D

Boiling Water Reactor



BWR/6 FUEL ASSEMBLIES & CONTROL ROD MODULE

> 1.TOP FUEL GUIDE 2.CHANNEL FASTENER **3.UPPER TIE** PLATE 4.EXPANSION SPRING **5.LOCKING TAB** 6.CHANNEL 7.CONTROL ROD BFUEL ROD 9.SPACER 10.CORE PLATE ASSEMBLY 11.LOWER TIE PLATE 12 FUEL SUPPORT PIECE **13 FUEL PELLETS** 14.END PLUG 15.CHANNEL SPACER 16.PLENUM SPRING

GENERAL 🎲 ELECTRIC

PWR and BWR Fuel Variants Advanced Cladding

- Zirlo (Zr-Nb alloy) and Duplex (layered) for PWRs
- Tweak Zircaloy composition for BWRs
- Moving to more smaller elements
- Mixed-oxide (MOX) fuel
 - U- 5-8% Pu
 - Already being done in US (weapons Pu) and elsewhere
 - Mixed actinides from advanced reprocessing
 - U-Np-Pu: modest extension of U-Pu fuel technology
 - Am-Cm: a challenge, probably requires targets
- More PWR fuel gradation and burnable poisons
- Thorium oxide fuel

Fast Breeder Reactor

- Dimensions: hexagonal, H=4-5.5m, W (flats)= 10-20 cm; HM height ~2m
- Weight: ~60 kgHM, ~65 kg MOX, ~135 kg hardware (core plus axial blanket)
 - Hardware: Stainless steel
 - Mostly wire wrap for pin spacing
- Fuel element array: 200-300 pins
- Fuel element size: 0.6-0.9 cm OD, H= 4-5m
- Enrichment: 15-30% Pu
- Blanket: All depleted UO₂
 - Fewer, larger diameter elements

Fast Reactor Fuel Assembly



Fast Reactor Fuel Variants

- Designs not settled: considerable variation in number of elements, dimensions, and weights possible
- Reduce breeding/conversion ratio to achieve net destruction of transuranics
 - Eliminate fertile blankets in favor of non-fertile neutron reflectors (e.g., stainless steel)

– Inert matrix (e.g., ZrO₂) fuel

• Carbide, nitride, or metal fuel instead of oxide

Metal-Clad Fuel Fabrication







Machining facility





MOX Fuel Fabrication

- The same as UO₂ fuel manufacture however....
 - Need to make sure the oxides are very homogenous and have required proportions
 - MOX fuel fabrication from powder usually dilutes a highconcentration "master blend" to the proper Pu enrichment
- Mixed-oxide (MOX): mixtures of actinide oxides
 - Conventionally, U-Pu
 - Advanced: U-Pu-Np, Am-Cm, combinations
- Conventional MOX fuels are being tested in the US but are currently being produced elsewhere
 – Facility using weapons Pu is being built at SRS

MOX Fuel Fabrication



Sol-Gel/Sphere-Pac Fabrication

- Used to prepare oxide fuels (U and MOX) without mixing powders, grinding, and dust
- Involves two steps:
 - Gelation: form kernels of U, Pu, and/or Th oxides by forming spheres
 - Based on ammonia precipitation of hydrated heavy metal oxides
 - 30, 300, and 1200µm optimal for Sphere-Pac (85% T.D.)
 - External gelation seems to be the current preference
 - Spheres are then washed and dried at ~200C
 - Sphere-Pac: Calcine and then sinter the spheres and load them into clad tubes
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Sol-Gel/Sphere-Pac Fabrication



Metal Fuel Fabrication

- Begin with a furnace containing a molten mixture of actinides, some fission products, and alloying constituents
- Make a mold having an array of quartz tubes
- Insert mold in furnace, seal, and evacuate
- Lower mold into melt and increase pressure to force metal melt into tubes
- Raise mold, cool, break to yield metal fuel "pellets" ~0.5m long.
- Insert in metal clad as with oxide fuels

Metal Fuel Fabrication





Fabrication Scrap Recycle

- All fabrication processes internally recycle offspecification fuel
 - Essentially dissolution and U/Pu/MOX purification steps used in a reprocessing plant
 - Off-spec metal fuels simply go back into the melt furnace or electrorefiner

Graphite-Based Fuels

HTGR Prismatic Fuel

- Dimensions: hexagonal, H=0.8m, 0.36m (flats)
- Weight: 5-7 kgU, 5.5-7.5 kg UO₂
 - Hardware 126 kg C (mostly graphite), 4 kg SiC
- ~1000 blocks for 600 MW(t) reactor
- Fuel element array: 210 on a triangular pitch – 108 Coolant channels
- Fuel element size: 1.3 cm OD, H=0.8m
 - Contains 14-15 "compacts" with 350-500µm TRISO particles
- Enrichment: 8-20%
- May have separate B₄C burnable poison rod[®]

HTGR Prismatic Fuel



HTGR Prismatic Fuel



Pebble Bed Reactor Fuel

- Dimensions: Spherical, D= 6.0 cm
- Weight: 9 g U, 10 g UO₂
 "Hardware" 194 g C (mostly graphite), ~6 g SiC
- ~360,000 pebbles for 400 MW(t) reactor
- Fuel element array: random pile
- Fuel element size:
 - 900µm TRISO particle
 - ~15,000 particles per pebble
- Enrichment: 7-10%



Pebble Bed Reactor



Graphite Fuel Fabrication

Graphite Fuel Fabrication



Spent Fuel Characteristics

Effects of Neutron Irradiation

- Elemental and isotopic composition changes
 - Actinides fission to produce energy
 - Rule-of-thumb: Fissioning 1% of actinides = 10 GWd/MT
 - Fission Products produced in amounts equal to actinides destroyed
 - Irradiation produces neutron capture products
 - Actinides: U-236, Np, Pu, Am, Cm, U-233
 - Hardware and fuel matrix: Activation products
 - Major constituents yield ⁹³Zr, ⁶⁰Co, etc.
 - Important trace contaminants: U (transuranics), Li (³H), N (¹⁴C)
- Physical changes: fuel swelling/cracking, clad embrittlement, fission gas release to plenum

How much burnup?

PWRs/BWRs

- Now 40-50 GWd/MTHM
- Climbing but enrichment challenges
- Fast reactors
 - Hope for 100+
 GWd/MTHM

Graphite-fueled reactors

Hope for 100+ GWd/MTHM





What are the impacts?

- Complex mix of elements to be considered in separations (~entire periodic table)
- Greatly increased radioactivity
 - Gamma-rays/x-rays
 - Neutrons: spontaneous fission and (α,n)
 - Alpha (helium nucleus)
 - Beta (electron)
- Impacts
 - Penetrating radiation \rightarrow need radiation shielding
 - Particles \rightarrow material damage
 - Decay heat \rightarrow provisions for heat removal

Decay Heat, PWR, 33 GWd/MT



Backup Slides

Pressurized Water Reactor



Boiling Water Reactor



Sodium-Cooled Fast Reactor







Very-High-Temperature Reactor

Very-High-Temperature Reactor



Gas-Cooled Fast Reactor



Lead-Cooled Fast Reactor



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Molten Salt Reactor



Molten Salt Reactor Fuel



Supercritical-Water-Cooled Reactor

Supercritical-Water-Cooled Reactor



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Pebble-Bed Modular Reactor



Canada Deuterium-Uranium Reactor



CANDU Calandria Schematic



CANDU Calandria Photo



CANDU Fuel Assembly



CANDU Online Refueling

